NANDAUER EUROPE

Improvements in neutron dosimetry at nuclear power plants

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The most widely adopted techniques world-wide for occupational neutron personal dosimetry are:

- Solid State Nuclear Track detectors using polycarbonate (CR-39) to detect proton recoils (for intermediate and fast neutrons) or alpha products (from ¹⁰B(n,α)⁷Li reactions in a Boron rich irradiator) for thermal neutrons;
- Albedo dosemeters which incorporate a thermal neutron sensitive detectors (TLD) or more recently using optically stimulated luminescence (OSL) detectors (aluminium oxide coated with ⁶Li₂CO₃)) which measure the reflected (albedo) thermal neutrons backscattered from the body.

Both techniques have advantages and disadvantages in practical situations which have been discussed widely and are summarised in this poster. LANDAUER has been providing personal neutron dosimetry services to clients operating nuclear power plants for a number of years. Recently we have worked with a number of our clients to improve the reliability and accuracy of personal neutron dosimetry measurements. This has included: • Assessment of the extent of doses to workers in nuclear power plants (typically 99.7% receive less than 1 mSv);

- Improvements to the dosemeters and their analysis;
- Characterisation of environments in which neutron exposure is of concern;
- Field calibration of dosemeters;
- Comparison with other types of dosemeters and manufacturers.

This paper will outline this work and draw together lessons learned and recommendations for the optimal assessment of personal dose in nuclear power plant environments.

Dosimetry systems

For personal monitoring of neutron dose-equivalent, LANDAUER uses different technologies.

Solid State Nuclear Track Detectors using polycarbonate (CR-39)

- Two types of Neutrak[®] offer a wide range of energy response: Neutrak J detects proton recoils (for intermediate and fast neutrons);
- Neutrak T also detects thermal neutrons, using the addition of a boron radiator, which produces alpha particles from the ${}^{10}B(n,\alpha)^7$ Li reaction.
- Either of these variants can be incorporated into LANDAUER's standard photon dosemeters: Luxel+®, to become Luxel+ Ja or Luxel+Ta, or InLight[®], designated InLight & Neutrak J or InLight & Neutrak T.

Albedo dosemeters

These are based on the InLight dosemeter and incorporate elements consisting of our standard Optically Stimulated Luminescence (OSL) detector material coated with ⁶LiCO₃.



Characterisation of environments in which neutron exposure is of concern

- Energy distribution
- Spectrum measurement using eg NPobe (BTI)
- Range from fission spectrum to 1/e to highly thermal
- Fluence estimation

Using ambient monitors eg: Wendi II, FHT 752, Studsvik 2202D or Eberline

Directional distribution

Calculation assuming Unidirectional and Isotropic ⇒ estimate of directional dose equivalent

Field calibration of InLight OSLN and Neutrak dosemeters

Example neutron calibration factors for the Albedo dosemeter in a US NPP are given in the table below⁽⁴⁾:

| Description | Neutron correction factor (NCF) |
|--|---------------------------------|
| Highest level in containment- direct line of sight to top of Steam Generator | 1.568 |
| Highest level in containment- direct line of sight to top of Steam Generator | 1.580 |
| Highest level in containment- direct line of sight to top of Steam Generator | 1.580 |
| Operating floor, mostly open grating above, some obstruction | 1.595 |
| Operating floor, mostly open grating above, some obstruction | 1.581 |
| Operating floor, mostly open grating above, some obstruction | 1.573 |
| Below grade, 12" of concrete above | 1.482 |
| Basement level | 1.608 |
| Basement level | 1.572 |
| Average | 1.571 |
| Standard deviation | 0.036 |
| Coefficient of Variation | 0.023 |



| | Advantages | Disadvantages |
|--------------|--|--|
| Neutrak | Wide energy response (Thermal to > 14MeV) No sensitivity to photons Less need for field characterisation Doses recorded permanently on the detectors | Limited dose response: 1 - 250 mSv for intermediate and fast neutrons 1 - 20 mSv thermal neutrons Variation in response as a function of angle. Assessment involves chemical etching in highly caustic solution, which is time consuming (typically 24 hours before a result can be obtained). |
| InLight OSLN | Easy/quick readout Ability to re-asses doses (OSL detectors) Re-use of dosemeters after annealing | Large dependence on response as a function of energy. Need for careful assessment of fields to establish accurate neutron correction factors (NCF) which range from 0.08 (high energy fields) to 5 (for moderated fields). Sensitivity to photons |

Experience

Extent of doses to workers in the nuclear industry

Comparison of the annual exposure to photons and neutrons (for 2009)⁽¹⁾.

| Number monitored | | Collective dose man.mSv | | | | |
|---------------------|--------|----------------------------|---------|-------|------|-------|
| monitorea | < 0.05 | <0.1 | 0.1 - 1 | 1 - 6 | >6 | |
| Neutron | | | | | | |
| 9050 | 97.3 | | 2.6 | 0.1 | 0.04 | 70 |
| Photon | | | | | | |
| 107050 | 66.0 | 28.4 | 5.1 | 0.5 | 0.5 | 42000 |

99.7% of workers are exposed to less than 1mSv.

Evolution of the utilisation of neutron dosemeters

Example neutron correction factors for InLight Alebedo (OSLN) and Neutrak dosemeters in French nuclear facilities⁽¹⁾:

| Work station | Comment | Neutron correction factor (NCF) | | | | | |
|--------------|----------------------|---------------------------------|-------------------------|--|--|--|--|
| | Comment | Albedo dosemeter | Neutrak CR-39 dosemeter | | | | |
| MELOX | Powder mixing | 0.67 | 1.57 | | | | |
| MELOX | Press | 0.84 | 1.58 | | | | |
| MELOX | Grinding and shaping | 0.71 | 1.65 | | | | |
| MELOX | Intermediate site | 0.74 | 1.60 | | | | |
| PWR site 1 | Various | 0.1 - 0.3 | | | | | |
| PWR site 2 | | 0.3 | | | | | |

Comparison of personal dosemeters in real fields⁽³⁾

| | Personal Dose Equivalent, Hp(10)/µSvh ⁻¹ | | | | | | |
|-------------------------|---|-------------------------------|--------------|--|--|--|--|
| | D ₂ O vapour recovery room | Heat transport Auxiliary room | Boiler room | | | | |
| Reference determination | 11.3 ± 1.2 | 73 ± 11 | 65 ± 9 | | | | |
| Thermo EPD N2 - Front | 77 ± 11 | 385 ± 54 | 127 ± 18 | | | | |
| DMC 2000 GN - Front | 46 ± 6 | 251 ± 11 | 135 ± 20 | | | | |
| Bubble - Front | 41.7 ± 2.2 | 219 ± 11 | | | | | |
| Albedo - Front | 24.2 ± 2.8 | 514 ± 72 | 140 ± 10 | | | | |
| HPA CR-39 – Front | 10.5 ± 2.0 | 76 ± 4 | 44 ± 3 | | | | |
| LANDAUER CR-39 - Front | 13.1 | 133 | 80 | | | | |
| LANDAUER FNTD - Front | 11.3 ± 1.4 | 40 ± 5 | 17 ± 2 | | | | |

Concluding remarks

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| | | | | | | | | |

Number of Neutrak dosemeters analysed by LANDAUER in Paris from $2005 - 2011^{(1)}$.



Improvements to the dosemeters and their analysis

Automatic counting for CR-39 Neutrak dosemeters

- Limit of detection for Neutrak dosemeter reduced to 0.1 mSv
- Independent performance test for compliance with ISO 21909:2005 carried out at the National Physical Laboratory (NPL) in the UK⁽²⁾.
- Development of Albedo OSL dosemeter



Whilst exposure to neutrons is limited to a relatively small number of employees and the overall dose is small in comparison with photon dose, it is nonetheless important to monitor individual's doses accurately. However, achieving the same level of accuracy as for photon doses is very difficult, if not impossible. Several methods have been developed for neutron dosimetry which have their own advantages and disadvantages. In the case of albedo dosemeters, neutron correction factors determined by an assessment of the of the radiation field are essential, but difficult to determine. However, the dosemeters are very easy to assess and in the case of LANDAUER's OSL technology can be re-assessed many times before being annealed and re-used. Solid State Nuclear Track Detectors, using CR-39 polycarbonate material, are less dependent on field characterisation, but take longer to assess and that assessment relies on chemical etching at high temperature in very caustic solution. There remains a wide divergence of the doses measured by various dosemeters in the same fields.

References

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